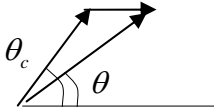


# Nuclear Reactor Theory

2008 년 1 학기

## Midterm Examination 1

April 3, 2008

1. Explain the following. Give proper units when required. (32 points, 4 each)
  - a. Microscopic and macroscopic cross section. (Unit)
  - b. The reason why the scalar flux is considered more important than the angular flux. (Unit)
  - c. Potential scattering and compound scattering
  - d. The reason for pronounced resonance absorption in heavy nuclei.
  - e. Classification of nuclear reactions with proper representation of each reaction in symbols.
  - f. Critical energy needed for fission and the reason why fissile nuclides are all odd-numbered.
  - g. Decay heat
  - i. Resonance self-shielding
2. Compute the following. Use  $0.6 \times 10^{24}$  for the Avogadro number. (12 points, 4 each)
  - a. Mean free path of a 1 keV neutron in water at the room temperature given that the total cross section of O-16 is almost constant at 4 barns in the intermediate energy. (4)
  - b. Average number of collisions required for a neutron to lose energy from 1 MeV to 1 eV by elastic scattering in a graphite medium. (4)
  - c. Energy recoverable in MWD by fissioning all nuclei in 1g of U-235.
3. Derive the following. (27 points, 5 each, indicated otherwise)
  - a. The relation between the cosines of the scattering angle in the lab and CM system:  $\mu = \frac{1 + A\mu_c}{\sqrt{1 + 2A\mu_c + A^2}}$ . 
  - b. Scattering kernel  $K(u' \rightarrow u)$  in case of isotropic scattering in CMS. Use the following relations for the derivation:  $\mu_c = \frac{1}{2A} \left( \frac{E}{E'} (A+1)^2 - (A^2 + 1) \right)$  and  $du = -\frac{dE}{E}$ . (6)
  - c. Energy dependence of Maxwellian flux spectrum. Start from the Boltzmann distribution,  $P(v_x, v_y, v_z) = C e^{-\frac{E}{kT}}$  where  $E = \frac{1}{2} m(v_x^2 + v_y^2 + v_z^2)$ . Find the final normalization constant. (6)
  - d. Slowing down density for the 1/E spectrum given by  $\phi(E) = \frac{S_0}{\xi \Sigma_s E}$  for a single isotope moderator region.
  - e.  $\int_{-\infty}^{\infty} e^{-x^2} dx = \sqrt{\pi}$ .
4. Plot schematically the following and answer associated questions. (12 points, 4 each)
  - a. Neutron spectrum in thermal reactors. Indicate clearly your axis labels. Provide approximate value for distinct energy region boundaries
  - b.  $\eta(E)$  for enriched fuel. Indicate approximate ticks on the y-axis. What is the consequence of this plot?
  - c. Fission spectrum in semi-log scale. Indicate the most probable and average fission neutron energies.

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5. Solve the following slowing down equation for a fuel and hydrogen mixture and answer the associated questions. (17 Points)

$$\Sigma_t(E)\phi(E) - \int_E^\infty \frac{\Sigma_s^H(E')\phi(E')}{E'} dE' = \chi(E)s_0 \quad \text{where} \quad \Sigma_t(E) = \Sigma_a^F(E) + \Sigma_s^H(E)$$

- Obtain the solution for the energy region below the fission source range. (9)
- Give the non-absorption probability and explain its variation during slowing down with a drawing. (4)
- What is the virgin or uncollided flux? (4)